

ASME OM-2-2024

Component Testing Requirements at Nuclear Facilities

AN AMERICAN NATIONAL STANDARD



The American Society of
Mechanical Engineers

ASME OM-2-2024

Component Testing Requirements at Nuclear Facilities

AN AMERICAN NATIONAL STANDARD



**The American Society of
Mechanical Engineers**

150 Clove Road • Little Falls, NJ • 07424 USA

Date of Issuance: October 30, 2024

This Code will be revised when the Society approves the issuance of a new edition.

This code or standard was developed under procedures accredited as meeting the criteria for American National Standards. The standards committee that approved the code or standard was balanced to ensure that individuals from competent and concerned interests had an opportunity to participate. The proposed code or standard was made available for public review and comment, which provided an opportunity for additional public input from industry, academia, regulatory agencies, and the public-at-large.

ASME does not “approve,” “certify,” “rate,” or “endorse” any item, construction, proprietary device, or activity. ASME does not take any position with respect to the validity of any patent rights asserted in connection with any items mentioned in this document, and does not undertake to insure anyone utilizing a standard against liability for infringement of any applicable letters patent, nor does ASME assume any such liability. Users of a code or standard are expressly advised that determination of the validity of any such patent rights, and the risk of infringement of such rights, is entirely their own responsibility.

Participation by federal agency representatives or persons affiliated with industry is not to be interpreted as government or industry endorsement of this code or standard.

ASME accepts responsibility for only those interpretations of this document issued in accordance with the established ASME procedures and policies, which precludes the issuance of interpretations by individuals.

The endnotes and preamble in this document (if any) are part of this American National Standard.



ASME Collective Membership Mark

All rights reserved. “ASME” and the above ASME symbol are registered trademarks of The American Society of Mechanical Engineers. No part of this document may be copied, modified, distributed, published, displayed, or otherwise reproduced in any form or by any means, electronic, digital, or mechanical, now known or hereafter invented, without the express written permission of ASME. No works derived from this document or any content therein may be created without the express written permission of ASME. Using this document or any content therein to train, create, or improve any artificial intelligence and/or machine learning platform, system, application, model, or algorithm is strictly prohibited.

The American Society of Mechanical Engineers
150 Clove Road, Little Falls, NJ 07424

Copyright © 2024 by
THE AMERICAN SOCIETY OF MECHANICAL ENGINEERS

CONTENTS

Foreword		vi
Committee Roster		vii
Correspondence With the OM Committee		x
Preface		xii
Cross-Referencing in ASME OM-2		xv
Section GR	General Requirements	1
GR-1	Introduction	1
GR-2	Owner Requirements	1
GR-3	Corrective Actions	3
GR-4	Records	3
Section DRD	Dynamic Restraint Devices	4
DRD-1	Introduction	4
DRD-2	Dynamic Restraint Device Requirements	4
DRD-3	Corrective Actions	8
DRD-4	Records	8
Section CP	Centrifugal Pumps	10
CP-1	Introduction	10
CP-2	Inservice Testing Requirements	10
CP-3	Corrective Actions	18
CP-4	Records	18
Section PDP	Positive Displacement Pumps	19
PDP-1	Introduction	19
PDP-2	Inservice Testing Requirements	19
PDP-3	Corrective Actions	26
PDP-4	Records	26
Section AOV	Air-Operated Valves	28
AOV-1	Introduction	28
AOV-2	Inservice Testing Requirements	28
AOV-3	Corrective Actions	31
AOV-4	Records and Test Plans	31
Section CV	Check Valves	32
CV-1	Introduction	32
CV-2	Inservice Testing Requirements	32
CV-3	Corrective Actions	35
CV-4	Records	35
Section HOV	Hydraulically Operated Valves	36
HOV-1	Introduction	36
HOV-2	Inservice Testing Requirements	36

HOV-3	Corrective Actions	39
HOV-4	Records	39
Section MOV	Motor-Operated Valves	40
MOV-1	Introduction	40
MOV-2	Inservice Testing Requirements	40
MOV-3	Corrective Actions	42
MOV-4	Records	43
Section MV	Manual Valves	44
MV-1	Introduction	44
MV-2	Inservice Testing Requirements	44
MV-3	Corrective Actions	45
MV-4	Records	45
Section PAV	Pyrotechnic-Actuated Valves	46
PAV-1	Introduction	46
PAV-2	Inservice Testing Requirements	46
PAV-3	Corrective Actions	47
PAV-4	Records	47
Section PRD	Pressure Relief Devices	48
PRD-1	Introduction	48
PRD-2	Inservice Testing Requirements	48
PRD-3	Corrective Actions	54
PRD-4	Records	55
Section SOV	Solenoid-Operated Valves	56
SOV-1	Introduction	56
SOV-2	Inservice Testing Requirements	56
SOV-3	Corrective Actions	59
SOV-4	Records	59
Section VLT	Valve Leak Testing	60
VLT-1	Introduction	60
VLT-2	Inservice Testing Requirements	60
VLT-3	Corrective Actions	61
VLT-4	Records	61
Section GL	Glossary	62
GL-1	Introduction	62
GL-2	Terms and Definitions	62
Section REF	References	64
 Figures		
CP-2.2.1-1	Centrifugal Pump Vibration Limits	14
PDP-2.2-1	Positive Displacement Pump Vibration Limits	23
 Tables		
GR-2.6-1	Test Frequency and Time Between Tests	3
DRD-2.2.5-1	Visual Examination Table	6
CP-2.1.1-1	Inservice Test Parameters for Centrifugal Pumps	10

CP-2.1.6.1-1	Required Instrument Accuracy — Hydraulic	11
CP-2.1.6.1-2	Required Instrument Accuracy — Mechanical	11
CP-2.2.1-1	Centrifugal Pump Test Acceptance Criteria	13
CP-2.2.2-1	Vertical Line Shaft Pump Test Acceptance Criteria	15
PDP-2.1.1-1	Inservice Test Parameters for Positive Displacement Pumps	19
PDP-2.1.6.1-1	Required Instrument Accuracy — Hydraulic	20
PDP-2.1.6.1-2	Required Instrument Accuracy — Mechanical	20
PDP-2.2-1	Positive Displacement Pump (Except Reciprocating) Test Acceptance Criteria	22
PDP-2.2-2	Reciprocating Positive Displacement Pump Test Acceptance Criteria	22
CV-2.4.4-1	Maximum Intervals for Use When Applying Internal Extensions	34
PRD-2.6.2.3-1	Seat Tightness Testing Methods for Pressure Relief Devices	54

FOREWORD

Inservice testing (IST) is used at nuclear facilities to examine, test, and monitor pumps, valves, and dynamic restraint devices. ASME OM-2, Component Testing Requirements at Nuclear Facilities, is a component code intended to be used for IST at nuclear facilities of various designs. ASME OM, Operation and Maintenance of Nuclear Power Plants, provides requirements for the performance of IST at water-cooled nuclear power plants. The purpose of IST is to test, examine, and monitor pumps, valves, and dynamic restraint devices to ensure the operational readiness of the component to perform its specified functions. Both ASME OM and ASME OM-2 accomplish this by trending degradation so that such degradation can be detected and remedied prior to the component being incapable of performing its specified functions. ASME prepared ASME OM specifically for light water reactors (LWRs). Additionally, ASME made several accommodations in ASME OM in the testing of components because many components cannot be fully tested with the nuclear power plant online. A reason for this is that ASME prepared ASME OM after many of those plants were built and operating.

With the expansion and evolution of the nuclear industry to facilities that are significantly different than the currently operating LWRs, ASME recognized that another code, ASME OM-2, for IST of components in new and advanced reactors was needed. ASME designed ASME OM-2 with the basic prerequisite that components that are to be part of the IST program for these new facilities are appropriate for the functions that they are to provide, that they are correctly sized and specified for the parameters of the system in which they are to be installed and operating, and that the materials of their construction are compatible with the system fluid conditions, pressures, temperature, and chemistry.

As a prerequisite, components that are to be in the IST program shall have been qualified by ASME QME-1, prior to their installation, or by a qualification method justified by the Owner to the applicable regulatory authority. At the time of preparing ASME OM-2, ASME prepared a reformatted edition of ASME QME-1 to allow its more effective application for nuclear facilities with various designs. As part of that qualification, the Owner is to provide the parameters of the component that need to be periodically monitored to ensure the operational readiness of the component to perform its specified functions. In addition to the periodic and condition-monitoring frequencies specified in this Code, manufacturer-specified inservice activities and associated frequencies shall be met, or alternatives justified by the Owner, and, if required by the applicable regulatory authority, submitted for the regulator's review and acceptance.

With the large number of different types of reactor systems being planned, and those that will be developed in the future, ASME OM-2 does not identify the specific components and specified functions that are to be tested in accordance with this Code. The selection of those specific components and specified functions is required to be completed by agreement between the system and facility designers, the component manufacturer, and the applicable regulatory authority. Once identified, those components must comply with ASME OM-2 unless an alternative is justified by the Owner and, if required by the applicable regulatory authority, submitted for the regulator's review and acceptance.

This Code does not include specific requirements for the application of risk insights. An applicant may use risk insights that takes into consideration the reactor design and planned operation in proposing its IST Program Plan for review and acceptance as required by the applicable regulatory authority.

The ASME Committee on Operation and Maintenance (OM Committee) of Nuclear Power Plants is tasked to ensure that standardized component test requirements contain the general and specific requirements necessary for those components that are selected to be tested. ASME OM-2 is one of these OM Committee codes available for inservice testing of components in nuclear facilities. While ASME prepared ASME OM-2 with nuclear facilities as the focus, non-nuclear facilities may use ASME OM-2 for IST of components in their facilities.

This publication, the 2024 edition of Operation and Maintenance of Nuclear Power Plants, was approved by the ASME Board on Nuclear Codes and Standards. ASME OM-2-2024 was approved by the American National Standards Institute on October 11, 2024.

ASME COMMITTEE ON OPERATION AND MAINTENANCE OF NUCLEAR POWER PLANTS

(The following is the roster of the committee at the time of approval of this Code.)

STANDARDS COMMITTEE OFFICERS

C. N. Pendleton, *Chair*
A. Cardillo, *Co-Vice Chair*
M. Gowin, *Co-Vice Chair*
O. Martinez, *Staff Secretary*

STANDARDS COMMITTEE PERSONNEL

T. Basso , Nuclear Energy Institute, Inc.	W. J. Roit , GE Hitachi Nuclear Energy
R. Binz IV , Retired	T. P. Ruggiero , Retired
A. Cardillo , Nuscale Power	T. G. Scarbrough , U.S. Nuclear Regulatory Commission
S. D. Comstock , GSE Solutions	G. E. Schinzel , Retired
D. A. Cruz , Arizona Public Service, Co.	F. Setzer , Electric Power Research Institute, Inc.
K. DeWall , Retired	B. Shank , Dominion Energy, Inc.
G. W. Doody , Retired	J. D. Stumb , Dominion Energy, Inc.
M. L. Garcia Heras , Westinghouse Electric Co., LLC	P. W. Turrentine , Element Materials Technology
C. A. Glass , Iddeal Solutions, LLC	S. M. Unikewicz , TerraPower
M. Gowin , Tennessee Valley Authority	B. J. Voll , Sargent & Lundy, LLC
W. Justice , Retired	G. Weiss , Constellation
R. Lippy , Retired	S. Bailey , <i>Alternate</i> , U.S. Nuclear Regulatory Commission
T. Loebig , Westinghouse Electric Co., LLC	R. S. Hartley , <i>Contributing Member</i> , Retired
D. Ludovisi , Sargent & Lundy, LLC	A. Dermenjian , <i>Honorary Member</i> , Retired
O. Martinez , The American Society of Mechanical Engineers	J. J. Dore , <i>Honorary Member</i> , GSE True North
M. McGaha , Iddeal Solutions, LLC	R. Emrath , <i>Honorary Member</i> , Retired
B. Moenkedick , Southern Nuclear Operating Co., Inc.	J. H. Ferguson , <i>Honorary Member</i> , Consultant
S. Norman , Enercon Services, Inc.	S. D. Hyten , <i>Honorary Member</i> , Retired
G. R. Palmer , Palmer Group International, LLC	B. Parry , <i>Honorary Member</i> , Retired
D. Patel , Southern Nuclear Operating Co., Inc.	R. Rana , <i>Honorary Member</i> , Retired
C. N. Pendleton , Retired	C. W. Rowley , <i>Honorary Member</i> , The Wesley Corp.
T. Robinson , Nebraska Public Power District	B. Scott , <i>Honorary Member</i> , GSE True North
W. Rogers , Iddeal Solutions, LLC	J. S. G. Williams , <i>Honorary Member</i> , Retired

SUBCOMMITTEE ON DYNAMIC RESTRAINTS

S. A. Norman , <i>Chair</i> , Enercon Services, Inc.	S. L. McCormick , Tennessee Valley Authority
T. D. Canter , <i>Vice Chair</i> , Duke Energy	G. R. Palmer , Palmer Group International, LLC
P. M. Peterson , <i>Vice Chair</i> , Iddeal Solutions, LLC	M. Palmer , ASC Engineered Solutions
G. S. Bedi , U.S. Nuclear Regulatory Commission	M. A. Pressburger , Sargent & Lundy, LLC
D. P. Brown , Lake Engineering Co.	T. Robinson , Nebraska Public Power District
D. Dutrow , Lisega, Inc.	M. Shutt , Retired
S. T. Esposito , Espo Consulting	K. Asmundsson , <i>Contributing Member</i> , GSE True North
M. Etten-Bohm , Southern Nuclear Operating Co., Inc.	H. Koski, Jr. , <i>Contributing Member</i> , Retired
S. Fox , Nextera Energy	R. Labeaf , <i>Contributing Member</i> , Retired
N. B. Frank , American Electric Power	R. L. Portmann, Jr. , <i>Contributing Member</i> , Consultant
A. L. Hand , Energy-Northwest	R. E. Richards , <i>Contributing Member</i> , ASC Engineered Solutions
K. Harris , STP Nuclear Operating Co.	

SUBCOMMITTEE ON GENERAL REQUIREMENTS

S. D. Comstock, *Chair*, GSE True North
W. Rogers, *Secretary*, Iddeal Solutions, LLC
R. Binz IV, Retired
G. W. Doody, Retired
M. Friedman, Nextera Energy
M. L. Garcia Heras, Westinghouse Electric Co., LLC
C. A. Glass, Iddeal Solutions, LLC
M. Gowin, Tennessee Valley Authority
E. Kennedy, Entergy
M. McGaha, Iddeal Solutions, LLC

B. Moenkedick, Southern Nuclear Operating Co., Inc.
S. A. Norman, Enercon Services, Inc.
J. Parsons, General Electric Co.
D. Patel, Southern Nuclear Operating Co., Inc.
C. N. Pendleton, Retired
T. Robinson, Nebraska Public Power District
W. J. Roit, GE Hitachi Nuclear Energy
T. G. Scarbrough, U.S. Nuclear Regulatory Commission
J. D. Stumb, Dominion Energy, Inc.

SUBCOMMITTEE ON NEW REACTORS

A. Cardillo, *Chair*, Nuscale Power
R. Binz IV, *Vice Chair*, Retired
T. Basso, Nuclear Energy Institute, Inc.
S. D. Comstock, GSE True North
A. Garcia, Westinghouse Electric Co., LLC
M. R. Holbrook, Idaho National Laboratory
W. Justice, Retired
I. T. Kisel, Retired
G. M. Kopanakis, Sargent & Lundy, LLC

D. Ludovisi, Sargent & Lundy, LLC
J. Melito, GE Hitachi Nuclear Energy Americas, LLC
G. R. Palmer, Palmer Group International, LLC
W. R. Peebles, Jr., Retired
C. N. Pendleton, Retired
W. J. Roit, GE Hitachi Nuclear Energy
T. P. Ruggiero, Retired
T. G. Scarbrough, U.S. Nuclear Regulatory Commission
F. Setzer, Electric Power Research Institute, Inc.
S. M. Unikewicz, TerraPower

SUBCOMMITTEE ON PIPING SYSTEMS

B. J. Voll, *Chair*, Sargent & Lundy, LLC
G. Antaki, Becht Engineering Co., Inc.
C. Basavaraju, U.S. Nuclear Regulatory Commission
K. K. Fujikawa, Retired

R. Gilada, Retired
D. C. Hanes, Engineering Dynamics, Inc.
M. Kassar, Constellation
M. Trubelja, Pacific Gas and Electric Co.

SUBCOMMITTEE ON PUMPS

M. McGaha, *Chair*, Iddeal Solutions, LLC
B. Moenkedick, *Vice Chair*, Southern Nuclear Operating Co., Inc.
D. Patel, *Secretary*, Southern Nuclear Operating Co., Inc.
R. Binz IV, Retired
G. W. Doody, Retired
M. Friedman, Nextera Energy
M. Gowin, Tennessee Valley Authority
R. S. Hartley, Retired
E. Hudson, Electric Power Research Institute, Inc.

W. Justice, Retired
I. T. Kisel, Retired
T. Loebig, Westinghouse Electric Co., LLC
C. N. Pendleton, Retired
T. Robinson, Nebraska Public Power District
T. P. Ruggiero, Retired
W. R. Tomlinson, Flowserve
B. Voll, Sargent & Lundy, LLC
Y. Wong, U.S. Nuclear Regulatory Commission

SUBCOMMITTEE ON RISK-INFORMED ACTIVITIES

D. Ludovisi, *Chair*, Sargent & Lundy, LLC
J. Bradford, Southern Nuclear Co.
R. Brightup, Constellation
M. L. Garcia Heras, Westinghouse Electric Co., LLC

G. E. Schinzel, Retired
B. Scott, GSE True North
A. Dermenjian, *Honorary Member*, Retired

SUBCOMMITTEE ON VALVES

D. Patel, *Chair*, Southern Nuclear Operating Co., Inc.
M. Gowin, *Vice Chair*, Tennessee Valley Authority
B. Moenkedick, *Secretary*, Southern Nuclear Operating Co., Inc.
M. Barlok, Tennessee Valley Authority
R. Binz IV, Retired
J. Bradford, Southern Nuclear Co.
S. D. Comstock, GSE True North
D. A. Cruz, Arizona Public Service Co.
W. E. Densmore, Westinghouse Electric Co., LLC
M. L. Garcia Heras, Westinghouse Electric Co., LLC
C. A. Glass, Iddeal Solutions, LLC
T. L. Julius, Fluor Corp.
W. Justice, Retired

S. R. Khan, Sargent & Lundy, LLC
R. R. Lack, NWS Technologies, LLC
C. Linden, Arizona Public Service Co.
K. Musante, PSEG-Nuclear
R. Rana, Retired
W. Rogers, Iddeal Solutions, LLC
T. G. Scarbrough, U.S. Nuclear Regulatory Commission
F. Setzer, Electric Power Research Institute, Inc.
B. Shank, V.C. Summar Nuclear Station
C. Smith, Duke Energy Carolinas, LLC
R. S. Smith, Tennessee Valley Authority
J. D. Stumb, Dominion Energy, Inc.
G. Weiss, Constellation

SUBGROUP ON AIR-OPERATED VALVES

M. Barlok, *Chair*, Tennessee Valley Authority
C. Linden, *Vice Chair*, Arizona Public Service Co.
C. Shinafelt, *Secretary*, Nextera Energy
S. A. Anderson, Duke Energy Carolinas, LLC
K. F. Beasley, Kalsi Engineering, Inc.
N. Camilli, Electric Power Research Institute, Inc.

T. Coyle, Constellation
W. Fitzgerald, Creative Juices Consulting, LLC
D. Lurk, Southern Nuclear Operating Co., Inc.
K. Musante, PSEG-Nuclear
F. Setzer, Electric Power Research Institute, Inc.
Y. Wong, U.S. Nuclear Regulatory Commission

SUBGROUP ON CHECK VALVES

D. A. Cruz, *Chair*, Arizona Public Service Co.
M. Rain, *Secretary*, Flowserve Corp.
V. T. Ataman, AREVA NP, Inc.
W. E. Densmore, Westinghouse Electric Co., LLC
L. I. Ezekoye, Ezekoye Engineering Services
N. Hansing, U.S. Nuclear Regulatory Commission
E. Noviello, Crane Nuclear, Inc.
J. Pilon, Constellation

A. Ritz, Constellation
W. Rogers, Iddeal Solutions, LLC
J. L. Sabina, Energy Nuclear Generation Co., LLC
M. Sleiman, Arizona Public Service
R. S. Smith, Tennessee Valley Authority
T. E. Thygesen, Retired
P. Twaddle, Constellation
M. Wills, Tennessee Valley Authority

SUBGROUP ON MOTOR-OPERATED VALVES

W. E. Densmore, *Chair*, Westinghouse Electric Co., LLC
D. A. Cruz, *Vice Chair*, Arizona Public Service Co.
C. Reynolds, *Secretary*, Constellation
K. DeWall, Retired
S. Hale, Crane Nuclear, Inc.
D. Lurk, Southern Nuclear Operating Co., Inc.
T. S. Neckowicz, Constellation

T. G. Scarbrough, U.S. Nuclear Regulatory Commission
C. Smith, Duke Energy Carolinas, LLC
J. E. Thilking, Flowserve Corp.
K. W. Trappett, Jr., Energy Northwest
T. A. Walker, Electric Power Research Institute, Inc.
J. Weseloh, Evergy
R. Yeardley, National Technical Systems, Inc.

SUBGROUP ON PRESSURE RELIEF DEVICES

R. R. Lack, *Chair*, NWS Technologies
B. R. Collier, *Secretary*, RC Engineering
P. M. Barnes, Sargent & Lundy, LLC
N. Camilli, Electric Power Research Institute, Inc.
S. D. Comstock, GSE True North
C. A. Glass, Iddeal Solutions, LLC
N. Hansing, U.S. Nuclear Regulatory Commission
S. P. Jansen, Emerson Automation Solutions
R. Lowery, Curtiss-Wright Flow Control Corp.
M. Mir, Bruce Power, Inc.

R. J. O'Neill, Electric Power Research Institute, Inc.
J. Pilon, Constellation
A. Ritz, Constellation
W. J. Roit, GE Hitachi Nuclear Energy
T. G. Scarbrough, U.S. Nuclear Regulatory Commission
C. Shepherd, DTE Energy
P. W. Turrentine, Element Materials Technology
P. Twaddle, Constellation
I. Wilson, Constellation

CORRESPONDENCE WITH THE OM COMMITTEE

General. ASME codes and standards are developed and maintained by committees with the intent to represent the consensus of concerned interests. Users of ASME codes and standards may correspond with the committees to propose revisions or cases, report errata, or request interpretations. Correspondence for this Code should be sent to the staff secretary noted on the committee's web page, accessible at <https://go.asme.org/OMcommittee>.

Revisions and Errata. The committee processes revisions to this Code on a continuous basis to incorporate changes that appear necessary or desirable as demonstrated by the experience gained from the application of the Code. Approved revisions will be published in the next edition of the Code.

In addition, the committee may post errata on the committee web page. Errata become effective on the date posted. Users can register on the committee web page to receive email notifications of posted errata.

This Code is always open for comment, and the committee welcomes proposals for revisions. Such proposals should be as specific as possible, citing the paragraph number, the proposed wording, and a detailed description of the reasons for the proposal, including any pertinent background information and supporting documentation.

Cases

(a) The most common applications for cases are

(1) to permit early implementation of a revision based on an urgent need

(2) to provide alternative requirements

(3) to allow users to gain experience with alternative or potential additional requirements prior to incorporation directly into the Code

(4) to permit the use of a new material or process

(b) Users are cautioned that not all jurisdictions or owners automatically accept cases. Cases are not to be considered as approving, recommending, certifying, or endorsing any proprietary or specific design, or as limiting in any way the freedom of manufacturers, constructors, or owners to choose any method of design or any form of construction that conforms to the Code.

(c) A proposed case shall be written as a question and reply in the same format as existing cases. The proposal shall also include the following information:

(1) a statement of need and background information

(2) the urgency of the case (e.g., the case concerns a project that is underway or imminent)

(3) the Code and the paragraph, figure, or table number

(4) the editions of the Code to which the proposed case applies

(d) A case is effective for use when the public review process has been completed and it is approved by the cognizant supervisory board. Approved cases are posted on the committee web page.

Interpretations. Upon request, the committee will issue an interpretation of any requirement of this Code. An interpretation can be issued only in response to a request submitted through the online Inquiry Submittal Form at <https://go.asme.org/InterpretationRequest>. Upon submitting the form, the inquirer will receive an automatic email confirming receipt.

ASME does not act as a consultant for specific engineering problems or for the general application or understanding of the Code requirements. If, based on the information submitted, it is the opinion of the committee that the inquirer should seek assistance, the request will be returned with the recommendation that such assistance be obtained. Inquirers can track the status of their requests at <https://go.asme.org/Interpretations>.

ASME procedures provide for reconsideration of any interpretation when or if additional information that might affect an interpretation is available. Further, persons aggrieved by an interpretation may appeal to the cognizant ASME committee or subcommittee. ASME does not "approve," "certify," "rate," or "endorse" any item, construction, proprietary device, or activity.

Interpretations are published in the ASME Interpretations Database at <https://go.asme.org/Interpretations> as they are issued.

Committee Meetings. The OM Standards Committee regularly holds meetings that are open to the public. Persons wishing to attend any meeting should contact the secretary of the committee. Information on future committee meetings can be found on the committee web page at <https://go.asme.org/OMcommittee>.

PREFACE

GENERAL

In 2022, the ASME OM Committee directed that two separately published ASME codes be considered. ASME OM-2, Component Testing Requirements at Nuclear Facilities, is the second published ASME OM Committee code. The first ASME OM Committee code is ASME OM, Operation and Maintenance of Nuclear Plants.

ASME prepared ASME OM-2 to specify provisions for IST programs that will be appropriate for all types of nuclear facilities. Components within the scope of ASME OM-2 include those that perform one of the following functions:

- (a) generate, allow, throttle, or isolate fluid flow
- (b) provide pressure relief
- (c) establish dynamic restraint to ensure the structural integrity of piping systems and their components

To simplify ASME OM-2 language, the components that perform these functions are referred to as pumps, valves, and dynamic restraint devices in ASME OM-2. However, the components performing these functions in certain new or advanced nuclear facilities might have significantly different designs than components performing those functions in current water-cooled reactors. The IST provisions in ASME OM-2 may be specified for application to components that perform the functions within the scope of ASME OM-2 for all types of nuclear facilities, regardless of the design of the components.

ORGANIZATION

ASME OM-2 has a General Requirements section followed by sections that address program and testing requirements for dynamic restraint devices, pumps, and valves. The last two sections are a glossary and a list of references, respectively.

- [Section GR](#), General Requirements
- [Section DRD](#), Dynamic Restraint Devices
- [Section CP](#), Centrifugal Pumps
- [Section PDP](#), Positive Displacement Pumps
- [Section AOV](#), Air-Operated Valves
- [Section CV](#), Check Valves
- [Section HOV](#), Hydraulically Operated Valves
- [Section MOV](#), Motor-Operated Valves
- [Section MV](#), Manual Valves
- [Section PAV](#), Pyrotechnic-Actuated Valves
- [Section PRD](#), Pressure Relief Devices
- [Section SOV](#), Solenoid-Operated Valves
- [Section VLT](#), Requirements for Valve Leak Testing
- [Section GL](#), Glossary
- [Section REF](#), References